



Continuing Nuclear Data Research for Production of Accelerator-Based Novel Radionuclides for Medical Use: A Mini-Review

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Nuclear data are important for production and medical application of a radionuclide. This brief review concentrates on nuclear reaction cross-section data. The availability of standardized nuclear data for accelerator-based production of medical radionuclides is outlined. Some new directions in radionuclide applications, for example, theranostic approach, bimodal imaging, and radionuclide targeted therapy, are considered and the status of relevant nuclear data is discussed. The current trends in nuclear data research using accelerators are elaborated. The increasing significance of intermediate energy accelerators in production of therapeutic radionuclides is emphasized.

Keywords: accelerator-produced radionuclides, decay properties, reaction cross sections, positron emitters, therapeutic radionuclides, theranostic approach, radionuclide targeted therapy

INTRODUCTION

Radionuclides find application in medicine, both in diagnosis and internal radiotherapy. Diagnostic investigations are carried out using short-lived radionuclides which emit either a single or major γ -ray of energy between 100 and 250 keV or a positron, the former in single photon emission computed tomography (SPECT) and latter in positron emission tomography (PET). In internal radionuclide therapy, on the other hand, radionuclides emitting low-range highly ionizing radiation, that is, α - or β^- -particles, conversion, and/or Auger electrons are used. A detailed knowledge of decay properties of a radionuclide is, therefore, essential to decide about its application. Furthermore, the radionuclidic purity and specific activity (defined as the radioactivity per unit mass of the material) play an important role in its medical application. Radionuclidic purity reduces the radiation dose due to impurities and higher specific activity ensures that the biological equilibrium is not disturbed and imaging is carried out at real molecular level. Those two parameters are controlled by knowledge of nuclear reaction cross sections, especially in production of radionuclides *via* charged particle-induced reactions [1, 2]. The level and type of radionuclidic impurity varies with the chosen reaction route and the projectile energy range effective in the target.

The positron emitters are generally produced at small-sized cyclotrons, the number of which worldwide is now increasing to about 1200 [3]. Attempts are presently also underway to produce several therapeutic radionuclides at cyclotrons/accelerators, using either the charged particle beam or secondary radiation, that is, neutrons and photons, generated at accelerators (for recent reviews, cf [4, 5]). Nuclear data play a continuing role in the development of radionuclides for applications. In this brief review, the availability of standardized data for production of medical radionuclides using

accelerators is outlined, some new directions in radionuclide applications are discussed, and the current trends in nuclear data research are described.

AVAILABILITY AND STATUS OF STANDARDIZED DATA

Experimental data are compiled in the international file EXFOR, managed by the IAEA. Radionuclides to be used for a suitable modality of the medical procedure demand well-established data. The decay data of medical radionuclides are compiled and constantly updated in the MIRD file of the Society of Nuclear Medicine (SNM) of United States. Decay data of all radionuclides are available in the Evaluated Nuclear Structure and Decay Data File (ENSDF) and neutron-induced reaction cross-section data in the Evaluated Nuclear Data File (ENDF/B-VIII.0). In contrast, the evaluation methodology for charged particle-induced reaction cross sections started developing only about 2 decades ago [2] under the umbrella of IAEA in the form of coordinated research projects (CRPs), with major contributions from Jülich, Debrecen, Brussels, and Obninsk. To date three CRPs have been completed. The first two were related to radionuclides commonly used in diagnosis (IAEA-TECDOC-1211, 2001) and internal therapy (IAEA-Technical Report-473, 2011). In the third CRP, some updates were done but also the production data of some novel radionuclides were standardized [6–8]. The initial work was rather empirical. Later, however, strong application of nuclear models was built in. All modern calculation codes are based on the statistical model, taking into account angular momentum, nuclear structure, and level density effects, and also incorporate pre-compound emission contributions. Each code reproduces the experimental data to a certain degree of success. The nuclear model calculations performed using the codes EMPIRE 3.2 and TALYS 1.95 are generally very successful in reproducing the experimental data up to about 50 MeV, if only a few nucleons are emitted. Deviations are observed when emission of many nucleons and complex particles is involved. The adjustable parameters of the nuclear model codes are fitted within their prescribed limits to reproduce the experimental data in the process of validation and evaluation. All evaluated data are available on the website of the IAEA [9]. Using theory-aided evaluation, some work has been also carried out outside the abovementioned CRPs (see for example [10–14]). The evaluation efforts have provided reliable standardized data for accelerator-based radionuclides routinely used in patient care (PET/SPECT imaging and internal radiotherapy), produced either directly or obtained through positron-emitting generator systems $^{68}\text{Ge}/^{68}\text{Ga}$ and $^{82}\text{Sr}/^{82}\text{Rb}$. Furthermore, data for a few novel and less commonly used radionuclides, for example, ^{64}Cu , ^{89}Zr , ^{124}I , ^{103}Pd , and ^{211}At , have also been standardized. Standardized data are also available for the $^{100}\text{Mo}(p,2n)^{99\text{m}}\text{Tc}$ reaction [10]. This is a very promising route for accelerator-based production of the most commonly used SPECT radionuclide $^{99\text{m}}\text{Tc}$, which is generally available *via* the fission-produced $^{99}\text{Mo}/^{99\text{m}}\text{Tc}$ generator system. However, further extensive work is needed to settle the question of impurities in cyclotron production of this

radionuclide. Similarly, for emerging radionuclides more experimental and evaluation work is necessary (see below).

NEW DIRECTIONS IN RADIONUCLIDE APPLICATIONS AND RELEVANT NUCLEAR DATA

For investigating slow metabolic processes, about 25 longer lived metallic positron emitters, termed as “nonstandard” positron emitters, have been developed [15]. Similarly, several metallic radionuclides emitting low-energy corpuscular radiation are in development for internal therapy (cf [1, 16]). In parallel, considerable chemical research has led to enhanced possibilities of labeling monoclonal antibodies (mAbs) and other versatile organo-metallic chemical complexes with radionuclides for targeted therapy (for reviews cf [17, 18]). Based on those advances, applications of radionuclides are enhancing today in the following directions:

- i) Radiolabeled monoclonal antibodies
- ii) Peptide receptor radiotherapy
- iii) Small molecules
- iv) Theranostic approach
- v) Bimodal imaging
- vi) Radioactive nanoparticles

Radiolabeled monoclonal antibodies: Monoclonal antibodies (mAbs) labeled with radionuclides are used in diagnosis and therapy of tumors. The application of mAbs labeled with positron emitters is called immunoPET (cf. [19]). To date, the most widely used radionuclide for this purpose is ^{89}Zr , but also ^{64}Cu has found application in preclinical studies. For immunotherapy, ^{90}Y , ^{111}In , and ^{225}Ac are potential candidates, for example, [^{90}Y]ibritumomab tiuxetan (Zevalin).

Peptide receptor radiotherapy (PRRT) makes use of peptide-based radiopharmaceuticals which can target different receptor systems like somatostatin receptors (SSTR), integrins, chemokine receptors, or the prostate-specific membrane antigen (PSMA). The most common approach uses octreotide derivatives labeled with ^{90}Y or ^{177}Lu to treat neuroendocrine tumors effectively, that is, [^{90}Y]Y-DOTATOC and [^{177}Lu]Lu-DOTATATE. Also [^{225}Ac]Ac-DOTATATE is under study. A further important molecular target is the chemokine receptor-4 CXCR-4. This can be targeted with Pentixafor and studies with ^{177}Lu -labeled Pentixafor in cancer patients are underway.

Small molecules: Besides radioiodinated pharmaceuticals, different radiometal-labeled small molecules have been developed to treat oncological diseases, for example, PSMA-617, a urea-based derivative with excellent affinity to PSMA overexpressing prostate cancer tumor cells. After binding to the target, the molecule is internalized and facilitates eradication of the tumor cells. Therefore [^{225}Ac]Ac-PSMA-617 and [^{177}Lu]Lu-PSMA-617 are under different clinical trials.

Theranostic approach entails a combination of diagnosis (molecular imaging) and internal radionuclide therapy (molecular targeted treatment). It makes use of two

radionuclides of the same element in the same chemical form, a positron emitter which allows quantitative diagnosis *via* PET and a therapeutic nuclide. Originally, the pair $^{86}\text{Y}/^{90}\text{Y}$ was developed (cf. [20]), but today several other theranostic pairs are also known, for example, $^{44}\text{gSc}/^{47}\text{Sc}$, $^{64}\text{Cu}/^{67}\text{Cu}$, $^{124}\text{I}/^{131}\text{I}$, and $^{152}\text{Tb}/^{161}\text{Tb}$ (for review cf [4]). Two other concepts also exist: 1) use of an analog pair of trivalent metallic radionuclides, that is, a positron emitter (^{44}gSc or ^{68}Ga) and a β^- (^{177}Lu) or an α -emitter (^{225}Ac). 2) use of a single radionuclide, emitting a β^- or α -particle as well as a low-energy γ -ray, which could be utilized for SPECT measurement to deliver data for dosimetry. Examples are ^{47}Sc , ^{67}Cu , ^{177}Lu , and ^{186}Re for β^- -therapy and ^{211}At , ^{223}Ra , ^{225}Ac , ^{213}Bi , $^{212}\text{Pb}/^{212}\text{Bi}$ generator, and ^{149}Tb for targeted alpha therapy (TAT). Several other alpha-particle emitters are also under consideration (cf [21]).

Bimodal imaging involves a combination of two organ-imaging techniques, for example, PET and magnetic resonance imaging (MRI) (for review cf [22]). From the viewpoint of PET, the major focus is on the elements Mn and Gd which are important contrast agents in MRI. The positron-emitting radionuclide $^{52\text{g}}\text{Mn}$ is of great current interest (cf [23, 24]). For Gd, no positron-emitting radionuclide is available and the use of ^{68}Ga -labeled Gd(III) complexes has been proposed.

Radioactive nanoparticles in medicine constitute a long-term perspective, provided the stability and toxicity problems are overcome. In animal and preclinical studies, considerable success has been reported (for review cf [25, 26]) but application in humans has yet to be demonstrated. The radionuclides ^{64}Cu and ^{68}Ga are widely used positron emitters for surface labeling of nanoparticles. The longer lived $^{52\text{g}}\text{Mn}$ and ^{89}Zr are also of great interest. For therapy, ^{186}Re and ^{225}Ac are considered to be very useful.

In the abovementioned new applications seven positron emitters, namely ^{44}gSc ($T_{1/2} = 3.9$ h), $^{52\text{g}}\text{Mn}$ ($T_{1/2} = 5.6$ d), ^{64}Cu ($T_{1/2} = 12.7$ h), ^{68}Ga ($T_{1/2} = 1.13$ h), ^{86}Y ($T_{1/2} = 14.7$ h), ^{89}Zr ($T_{1/2} = 78.4$ h), and ^{124}I ($T_{1/2} = 4.18$ d) are in great demand. Similarly, eight therapeutic radionuclides, namely ^{47}Sc ($T_{1/2} = 3.35$ d), ^{67}Cu ($T_{1/2} = 2.58$ d), ^{90}Y ($T_{1/2} = 2.7$ d), $^{117\text{m}}\text{Sn}$ ($T_{1/2} = 13.6$ d), ^{177}Lu ($T_{1/2} = 6.65$ d), ^{186}Re ($T_{1/2} = 3.78$ d), ^{223}Ra ($T_{1/2} = 11.4$ d), and ^{225}Ac ($T_{1/2} = 10.0$ d) are of great interest. For each radionuclide, the stringent criteria of purity and specific activity must be met.

The production of the listed positron emitters is generally carried out *via* the low-energy (p,n) reaction on a highly enriched solid target isotope. In the case of ^{68}Ga , the generator route is more commonly used. The status of the available cross-section data of the abovementioned positron emitters was reviewed and found to be generally good (cf. [1, 15]), except for ^{86}Y where considerable discrepancy existed.

In contrast to positron emitters, the therapeutic radionuclides ^{90}Y and ^{177}Lu are routinely produced using nuclear reactors. For the six other therapeutic radionuclides, it was shown (cf. [8, 16]) that the reactor production methods are not sufficient. For ^{47}Sc and ^{67}Cu , the required radionuclidic and chemical purity is not achieved and for ^{186}Re and $^{117\text{m}}\text{Sn}$, the specific activity is too low. The supply of ^{225}Ac and ^{223}Ra *via* reactor route is limited. Efforts are therefore underway to produce those radionuclides at

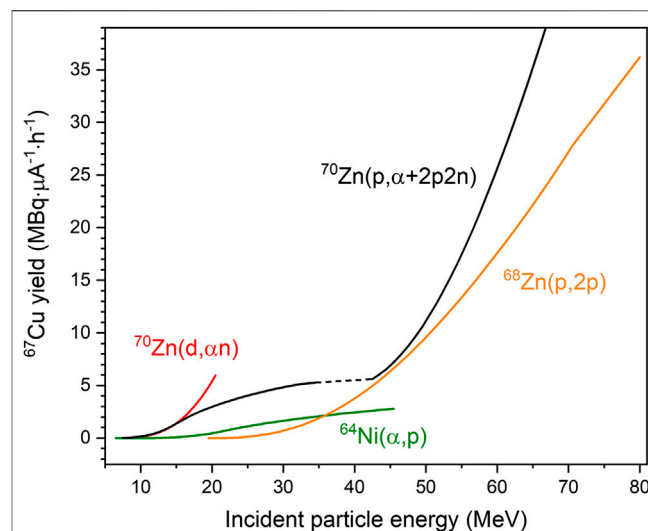


FIGURE 1 | Integral yield of ^{67}Cu , calculated from the excitation function of each reaction for an irradiation time of 1 h, shown as a function of incident particle energy. The dashed part of the curve for the $^{70}\text{Zn}(p,\alpha)^{67}\text{Cu}$ process is an extrapolation because no data exist.

accelerators, ^{186}Re at a small cyclotron and others at intermediate energy accelerators.

CURRENT NUCLEAR DATA ACTIVITIES

Nuclear modeling and theory-aided evaluations are continuing. The emphasis is, however, on new experimental studies; some of the most recent examples are given below.

Low-energy charged particle beam: Low-energy medical cyclotrons ($E_p \leq 18$ MeV; $E_d \leq 9$ MeV) are mainly utilized for the production of the standard positron emitters (^{11}C , ^{13}N , ^{15}O , and ^{18}F) using gas and liquid targets. Since solid targetry is not available, a new methodology for the production of some nonstandard positron emitters *via* the (p,n) reaction at those cyclotrons has emerged. It makes use of “solution targets”, especially for ^{44}gSc , ^{64}Cu , ^{68}Ga , ^{86}Y , ^{89}Zr , and $^{94\text{m}}\text{Tc}$ (for review cf. [27]). The product yield is low but sufficient for local use. This approach has, however, put an extra demand on chemical purification of the product as well as on the accuracy of the cross-section data near the threshold of the reaction (cf. [28, 29]). To improve the cross-section data of the $^{86}\text{Sr}(p,n)^{86}\text{Y}$ reaction, an accurate measurement was very recently completed [30]. The new results agree with the TALYS calculation and the discrepancy has been solved.

Intermediate energy charged particle beam: In the energy range up to 30 MeV, several routinely used radionuclides are conveniently produced, especially the SPECT radionuclides ^{67}Ga , ^{111}In , ^{123}I , and ^{201}Tl . At a higher proton energy up to 100 MeV, several radionuclides could be produced *via* (p,x) reactions, where x stands for multiparticle emission. Such accelerator facilities are now available at several places, for example Nantes (France), Legnaro (Italy), Moscow (Russia), Cape

Town (South Africa), Brookhaven (United States), and Los Alamos (United States). A recent example is the measurement of the excitation function of the $^{75}\text{As}(p,4n)^{72}\text{Se}$ reaction [31] for the production of ^{72}Se which is useful for preparing the β^+ -emitting generator system $^{72}\text{Se}/^{72}\text{As}$.

The intermediate energy range, however, is now being utilized more extensively for the production of therapeutic radionuclides. A recent cross-section measurement on the reaction $^{70}\text{Zn}(p,\alpha+2p2n)^{67}\text{Cu}$ [32] is very interesting. Although the experimental data are not reproduced by the model calculation (due to complexity of reaction and energy range), they are of great technical value. In **Figure 1** the yields of ^{67}Cu are shown. They were calculated from the excitation functions of relevant charged-particle-induced reactions using the standard activation equation [1]. It is an updated version of the diagram given earlier [16]. In order to minimize the impurities of ^{64}Cu (radioactive) and ^{65}Cu (stable, decreasing the specific activity of ^{67}Cu), the energy range $E_p = 80 \rightarrow 30$ MeV was found to be very suitable for the production of ^{67}Cu via the $^{68}\text{Zn}(p,2p)$ process. From the new results, it is concluded that protons of energy up to about 80 MeV are very suitable for the production of ^{67}Cu not only when isotopically enriched ^{68}Zn is used as target material [16, 33] but also if enriched ^{70}Zn would be the target, provided good radionuclidic purity is achieved. For production of the β^- -emitter ^{47}Sc (for review see [4]), neither the intermediate energy reaction $^{48}\text{Ti}(p,2p)^{47}\text{Sc}$ investigated earlier nor the $^{51}\text{V}(p,\alpha p)^{47}\text{Sc}$ process studied recently [34] appears to be successful because of the high level of the radionuclidic impurity ^{46}Sc . With regard to the conversion electron emitter $^{117\text{m}}\text{Sn}$, the production route $^{116}\text{Cd}(\alpha,3n)^{117\text{m}}\text{Sn}$ was established [11]. Yet, a new measurement suggests that the nuclear process $^{\text{nat}}\text{Sb}(p,xn)^{117\text{m}}\text{Sn}$ over the proton energy range of 30–90 MeV could also be potentially useful [35]. Regarding the α -particle emitters ^{225}Ac and ^{223}Ra , the present emphasis is on their production through proton-induced reactions on ^{232}Th , either directly or via the indirect routes $^{232}\text{Th}(p,x)^{225}\text{Ra} \xrightarrow{\beta^-} ^{225}\text{Ac}$ and $^{232}\text{Th}(p,x)^{227}\text{Th} \xrightarrow{\alpha^-} ^{225}\text{Ac}$, respectively, over the energy range up to 200 MeV. Several cross-section measurements exist (for review cf [8]) and studies on radionuclidic impurities, for example, about 0.3% ^{227}Ac in ^{225}Ac , as well as technical development are in progress. Besides its direct use, the radionuclide ^{225}Ac also serves as the parent of the α -emitter ^{213}Bi .

It should be mentioned that besides protons, intermediate energy deuterons and alpha-particles are also potentially useful for production of a few special radionuclides, for example, ^{103}Pd and ^{186}Re using deuterons and ^{211}At using alpha-particles. Furthermore, in recent years it has been demonstrated that the high-spin isomers of a few radionuclides, for example, $^{117\text{m}}\text{Sn}$ and $^{193\text{m}}\text{Pt}$, can be advantageously produced using alpha-particles [36]. They are useful in therapy because they emit low-energy conversion and Auger electrons. In general, however, the use of protons is preferred due to their easier availability and the resulting higher yields of the products.

High-energy charged-particle beam: The spallation process with high-energy protons ($E_p > 500$ MeV) combined with on-line mass separation was utilized at CERN in cooperation with the Paul Scherrer Institute (PSI) to produce some exotic radionuclides in the region of rare earths, especially ^{149}Tb ($T_{1/2} = 4.1$ h) and ^{152}Tb ($T_{1/2} = 17.5$ h). The former is a unique low-energy α -particle emitter suited for TAT. The latter is a theranostic PET partner. A new measurement gives cross sections for the formation of several terbium radioisotopes in the spallation of tantalum as a function of proton energy [37]. For more general use of the two radionuclides, however, development of alternative production methods, preferably using intermediate energy protons, are called for. Preliminary cross-section measurements on the reactions $^{155}\text{Gd}(p,4n)^{152}\text{Tb}$ and $^{152}\text{Gd}(p,4n)^{149}\text{Tb}$ are promising [38].

Use of photons and fast neutrons: The use of electron linear accelerator (LINAC) to deliver high-energy photons for the production of the therapeutic radionuclides ^{47}Sc and ^{67}Cu via the $^{48}\text{Ti}(\gamma,p)^{47}\text{Sc}$ and $^{68}\text{Zn}(\gamma,p)^{67}\text{Cu}$ reactions, respectively, is presently under investigation. The excitation functions are known (cf. IAEA-TECDOC-1178, 2000) but further improvement in the data is needed. The production for preclinical tests has been reported [39, 40]. In practice, however, GBq amounts of ^{67}Cu have been produced whereas the methodology for ^{47}Sc production is still developing. Some on-going nuclear data work deals with determination of spectrum-averaged cross sections and production yields [41, 42]. The production of ^{225}Ac via the $^{226}\text{Ra}(\gamma,n)^{225}\text{Ra} \xrightarrow{\beta^-} ^{225}\text{Ac}$ process is also being investigated. The estimated cross section is relatively high but the use of the radioactive target is a deterrent.

The use of accelerator-generated fast neutrons is also under investigation. In particular, a 30 or 40 MeVd(Be) or d(C) breakup neutron source is considered to be suitable for the production of a few radionuclides via the (n,p) or (n,np) reaction [43]. The estimated integral cross sections amount to a few mb [5]. The spallation neutrons also appear to be interesting for the formation of a few radionuclides [44]. However, extensive further nuclear data work is called for.

CONCLUDING REMARKS

Nuclear data play an important role in the production and medical application of accelerator-based radionuclides. The data for routine production of radionuclides for patient care have been standardized. For development of novel radionuclides, however, continuing data research is needed. The present thrust in medical application of radionuclides is directed toward PET studies using metallic positron emitters as well as toward targeted radionuclide therapy, preferably applying the theranostic approach. With tremendous developments in antibody labeling and organo-metallic complex formation chemistry, a big impetus has come to the field of theranostics. This is leading to an enhanced interest in accelerator-based production of radionuclides. The nonstandard positron emitters are produced at small-sized cyclotrons and now development of

production methodologies of many therapeutic radionuclides is shifting from nuclear reactors to intermediate energy accelerators. Another strategy is to utilize hard photons from powerful LINACs or fast neutrons from intermediate energy accelerators. With enhancing interest in versatile accelerators to produce novel medical radionuclides, the need of relevant nuclear data research is continuing.

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AUTHOR CONTRIBUTIONS

SQ developed the concept. MH elaborated the section on standardization of data. IS reviewed experimental data and calculated radionuclide yields. BN advised on new directions in medical applications. All authors contributed to the writing of the article and approved the submitted version.

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Conflict of Interest: The authors declare that the research was conducted in the absence of any commercial or financial relationships that could be construed as a potential conflict of interest.

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